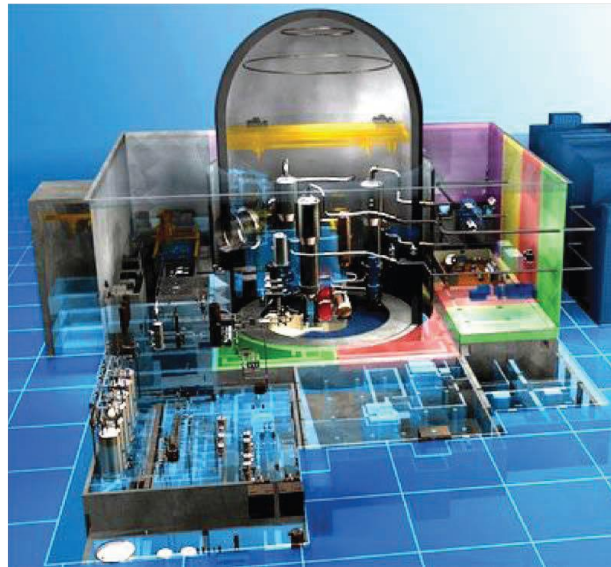




# NUCLEAR REACTOR SYSTEM

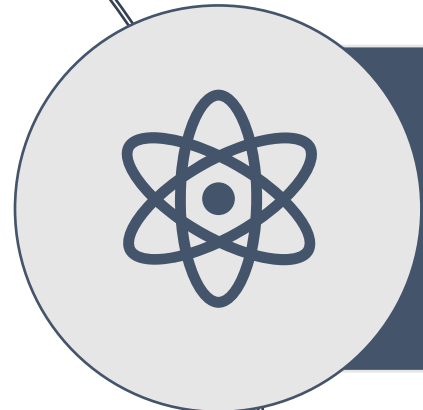


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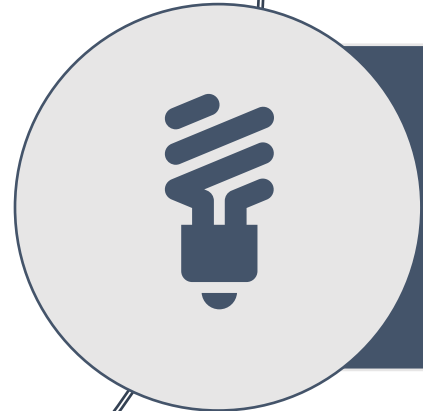




# Objectives of topic



**Introduce the important components in a nuclear reactor.**



**Students able to explain the functions for each of the components.**



# Reactor System

A **nuclear reactor** used for the purpose of **power generation** or for the **production of radioisotopes**.

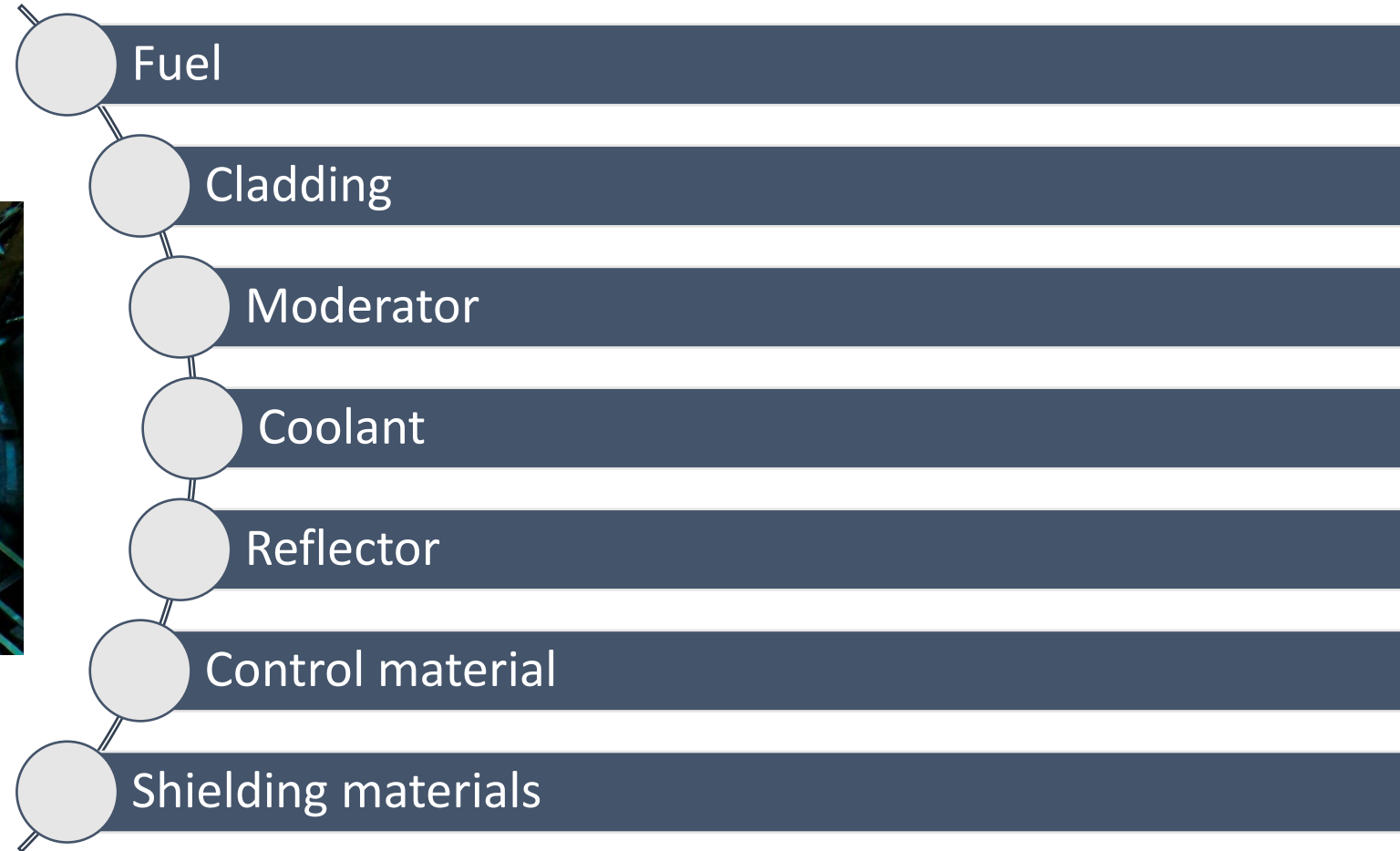
It is **defined** as a system wherein a **nuclear fission** is carried out as a **controlled chain reaction**

**Nuclear reactors** that utilize **thermal neutrons** to initiate and sustain fission are called **thermal reactors**

**Nuclear reactors** utilize fast neutrons are called **fast reactors**.

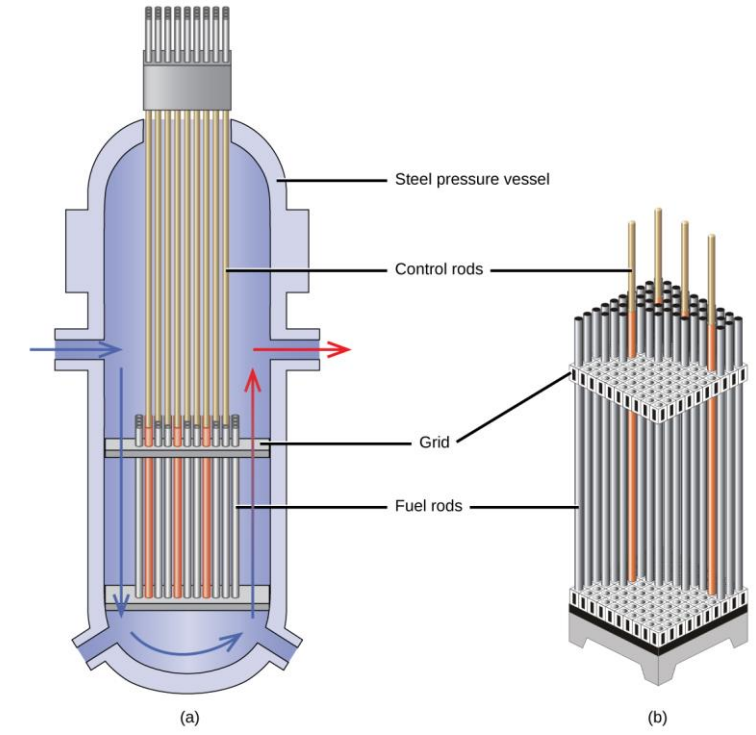
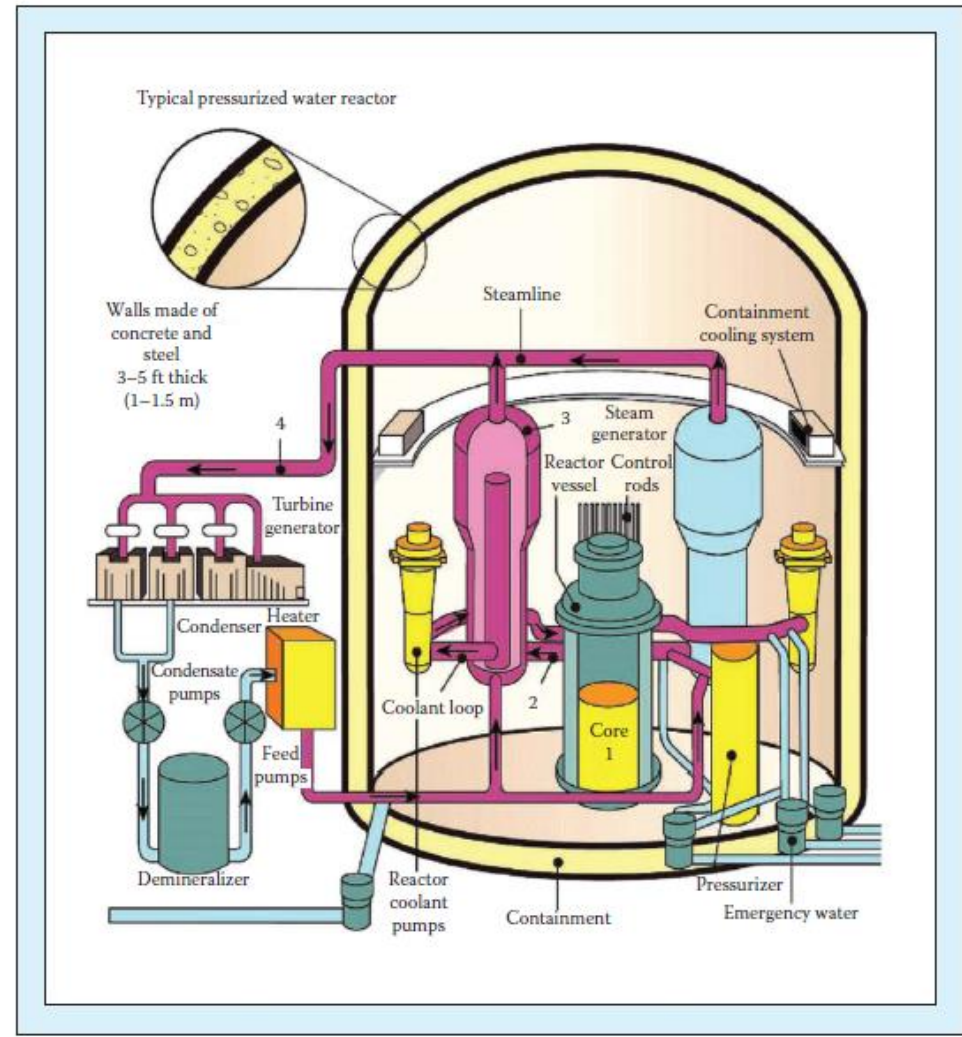


# Reactor System





# Overview of Nuclear Reactor Components



A typical PWR. Here, 1 denotes the core, 2 denotes the primary loop, 3 denotes the SGs, and 4 denotes the secondary loop. (Courtesy of the U.S. NRC.)



# Nuclear Fuel

Materials that undergo nuclear fission when bombarded with thermal neutrons are called **fissile** material and their respective nuclei called **fissile nuclei**.

The nuclei with higher  $\sigma_f/\sigma_\gamma$  are suitable as fuel as **U-235**, **Pu-239** and **U-233** have higher  $\sigma_f/\sigma_\gamma$  ratio. Hence, they can be used as fuel in **nuclear reactor**.

**U-235** is the only fissile nucleus that is available in nature, while **Pu-239** and **U-233** are produced in nuclear reactors.



# Nuclear Fuel

Cross sections for some nuclei for thermal neutrons (**Kinetic energy = 0.025 eV**)

Nucleus	Cross sections (b)			
	$\sigma_{\gamma}$	$\sigma_f$	$\sigma_T$	$\sigma_a$
U-235	95	586	700	681
Pu-239	270	752	1028	1022
U-238	2.73	11.8e-6	12.2	2.7

Cross sections for some nuclei for thermal neutrons (**Kinetic energy = 1 MeV**)

Nucleus	Cross sections (b)		
	$\sigma_{\gamma}$	$\sigma_f$	$\sigma_a$
U-235	0.11	1.2	2.7



# Nuclear Fuel

**Most** of the nuclear reactors for **power generation** use **uranium** as fuel.

**U-235** is not present in nature in **pure** form. It is present as a mixture with **U-238**, the major component present to the extent of **99.3 %**.

In other words, the **isotopic abundance** of natural U-235 in a mixture of **U-235** and **U-238** is only **0.7 %**.

In **most** reactors the **fuel** is in oxide form as **UO<sub>2</sub>**.

**Pu-239** is used as fuel in **Fast Reactors**.

**U-233** is proposed as fuel in **Advanced Heavy Water Reactors**.





# Nuclear Fuel Materials

The **nuclear reactors** in operation around the globe utilize **uranium** (or) **plutonium-based** materials as fuel.

**The fuel may be in the form:**

- Metal
- Metal Oxide
- Metal carbide
- Metal nitride



# Nuclear Fuel Materials

Properties	Implications
<b>Nuclear properties:</b> Absorption cross section, fission cross section, fission products, neutron production, energy released	Determines the mass of fissile required, volume of the core
<b>Thermal conductivity</b>	Indicates the ability of a material to transfer heat by thermal conduction.
<b>Melting point</b>	Defines the limiting power of a fuel element.
<b>Dimensional stability</b>	A measure of expansion of a material with increase in temperature.
<b>Chemical properties</b>	In case of cladding failure in terms of cracks, there will be direct contact between fuel and coolant.
<b>Coefficient of thermal expansion</b>	A measure of expansion of a material with increase in temperature.



# Oxide Fuels

**Oxide fuels** are

- UO<sub>2</sub> (Uranium oxide)**
- Mixed oxide (UO<sub>2</sub>-PuO<sub>2</sub>).**

The major **disadvantages** of oxide fuels:

- **Lower density** (leads to the requirement of a **larger core diameter**).
- **Lower thermal conductivity** (**poor transfer** of heat from the **centre** of the fuel pin to the edge of the pin and to the **cladding**).



# Metallic Fuels

**Metallic uranium** has **better thermal conductivity** compared to that of **UO<sub>2</sub>**. The **density** of metallic uranium is also **higher** than that of UO<sub>2</sub>.

The major **disadvantages** of oxide fuels:

- **Metallic uranium** undergoes a **change in crystalline** phase at **660 °C**.
- A substantial **change** in the **volume** of metal is experienced as a result of this phase change.
- This results in **damage** to the integrity of the **fuel elements**. Owing to this, metallic uranium is **not used** as fuel in **power reactors**



# Metal Carbide and Nitride Fuels

**Uranium carbide** and **uranium nitride** have **higher density** and **thermal conductivity** than uranium dioxide.

These fuels are **inferior** to **uranium dioxide** as far as the **compatibility** with **cladding**, **swelling** under **irradiation** and **retention** of fission gas are concerned.

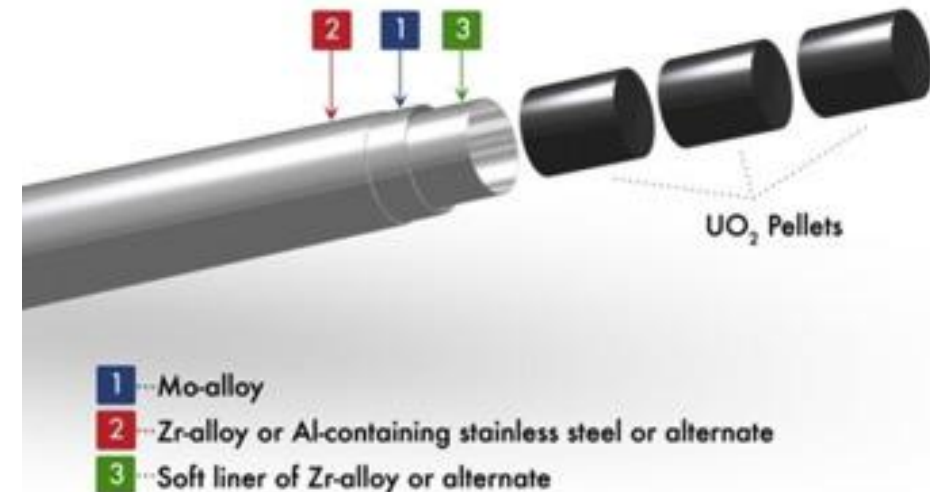
These were used in **test reactors** only.



# Cladding

## The **purpose** of cladding in a **nuclear reactor**

- Cladding gives the **physical configuration** by housing **fuel pellets**.
- Cladding retains the **fission products** and **prevents** direct contact between **coolant** and **fuel**.





# Cladding

The **material** of **choice** for **cladding** must:

- Possess **ductility**, impact **strength** and **creep** adequate enough to maintain cladding **unaltered** during the operation of the reactor.
- **Resistance** to **corrosion** by coolant must be high for a material to be used as cladding.
- Possess **high melting point**.
- Possess **high thermal conductivity**.
- **Not** be **damaged** due to sustained **neutron irradiation**.



# Cladding

The nuclear **properties** of clad material are

- must possess **low absorption** cross section for **neutrons**.
- **Aluminum, beryllium, magnesium** and **zirconium** have **low absorption** cross section for neutrons and possess **high melting point**.
- **Aluminum** is used in **small, research reactors**.
- **Magnox**, an alloy of Magnesium is used in **gas-cooled reactors**.





# Cladding

Continue..

- **Zircaloy**, the alloy of zirconium is used in most **thermal reactors**.
- These alloys (**Zircaloy-2** and **Zircaloy-4**) possess **good mechanical properties** and have superior **resistance to corrosion**.
- But **low absorption cross** section for neutrons was **not comprised** to a large extent.
- The requirement of **low absorption cross section** need not be satisfied for **fast reactors**.
- **Stainless steel** is used as cladding material in **sodium cooled fast reactors**.



## Moderator

**Moderation** with respect to nuclear reactors implies **slowing down** of **neutrons**.

**Neutrons** with **higher kinetic energy** (fast neutrons) are allowed to undergo **elastic scattering** with a **light nucleus** resulting in **reduction** in **speed** and change in **direction** of neutrons.

As a result, the **neutron energy** is brought down to **thermal levels (0.025 eV)**.

Comparing the **fission cross sections** of **U-235** with fast neutrons and thermal neutrons, it is clear that the probability of fission in **U-235** with **thermal neutrons** is **more** than with **fast neutrons**.



# Moderator

The **neutron energies** must be brought **down** or in other terms, the neutron needs to be **slowed down**.

The most widely used **moderators** are the one that contain **light nuclei** like

- **Hydrogen,**
- **Deuterium,**
- **Carbon**
- **Water (H<sub>2</sub>O, also called light water),**
- **Heavy water (D<sub>2</sub>O)**
- **Graphite**



# Coolant

The **purpose** of **coolant** is to **remove** the **heat liberated** during **fission** and to utilize the same for steam generation.

The **coolant supply** to a **nuclear reactor** must be **continued** even if the **chain reaction** has **ceased** and **reactor shut down**. This is to ensure the removal of **decay heat** and to **avoid** the **melting** of **reactor core**.

**Water** and **heavy water** are the most common liquid coolants in **thermal reactors**.

**Liquid sodium** is used as coolant in **fast reactors**.

**Helium** and **CO<sub>2</sub>** are used as gaseous coolants in **Gas-cooled reactors**.



# Reflector

The **purpose** of **reflector** in a nuclear reactor is to **reflect** the **neutrons escaping or leaving, back to the core.**

The essential **characteristics** required for a **reflector material**:

- **Low cross section** for neutron **capture** or **absorption**
- **High cross section** for **neutron scattering**
- **High energy loss** per collision event between **neutron** and **reflector**
- **Temperature** and **radiation stability**



# Reflector

Most **thermal reactors** use **water** as both the **moderator** and **reflector**.

The reflector in **PHWR** is the **heavy water**.

In **graphite-moderated reactors**, **graphite** acts as reflector.



# Control Materials

**Control materials** are essential for **regulating the power** of a reactor, apart from bringing about **rapid shut down** when required.

These are the materials that have **high tendency** to absorb neutrons through **neutron capture**.

These are the materials that possess **high  $\sigma$** .

**Boron** and **Cadmium** have high capture cross sections and hence are used as **control materials** in the form of **rods or plates**.



# Control Materials

**Control elements** are meant to **control** the reactor power through **absorption of neutrons**.

**Hafnium**, **silver-indium-cadmium** alloys and **boron carbide** are the widely used control materials.

**Hafnium** use as control material in **water-cooled reactors**. However, the **availability** of Hafnium is **limited** and hence it is **expensive**.

**Silver-indium-cadmium** alloys are **easily fabricated** must be **enclosed** in a **stainless steel** enclosure to protect the same from **corrosion**.

**Boron** has very **high absorption cross section** and **low cost** and must incorporated in a **metallic enclosure**.





# Shielding Materials

The **types of radiations** in a nuclear reactor are **neutrons**, **gamma**, **alpha** and **beta** radiation.

**Shielding** materials are **not required** for  **$\alpha$**  and  **$\beta$  radiation**.

For **absorption of neutron radiation**, a material with **low mass number** and **high cross section is suitable**. (Water)

**Iron** and **concrete** are good **neutron absorbers**. They can be easily fabricated as well.

Shield **gamma radiation** must be composed **of high mass number element**. (**Lead, iron and concrete**) for shielding

**Lead** is attractive due to its **low cost**, but its **lower melting point** is a **disadvantage**.



# Shielding Materials

## Other components

Apart the above four important components of a nuclear reactor, other components include **structural components** and **control systems**.



# Thank You

Stay safe!