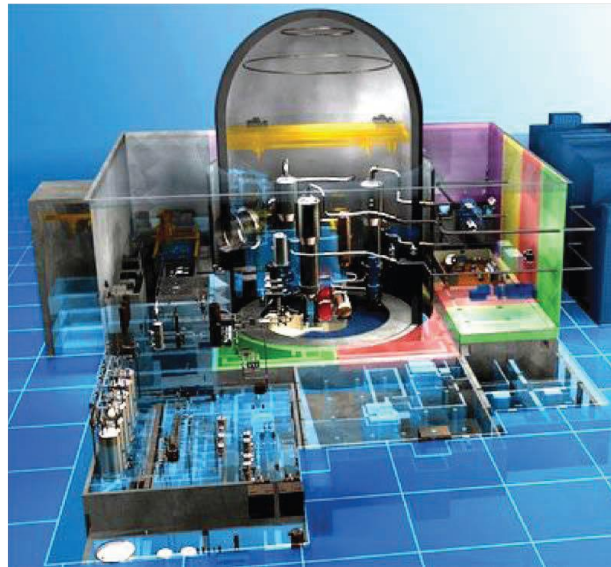




# Heat Conduction in Reactor Elements (Part 2)



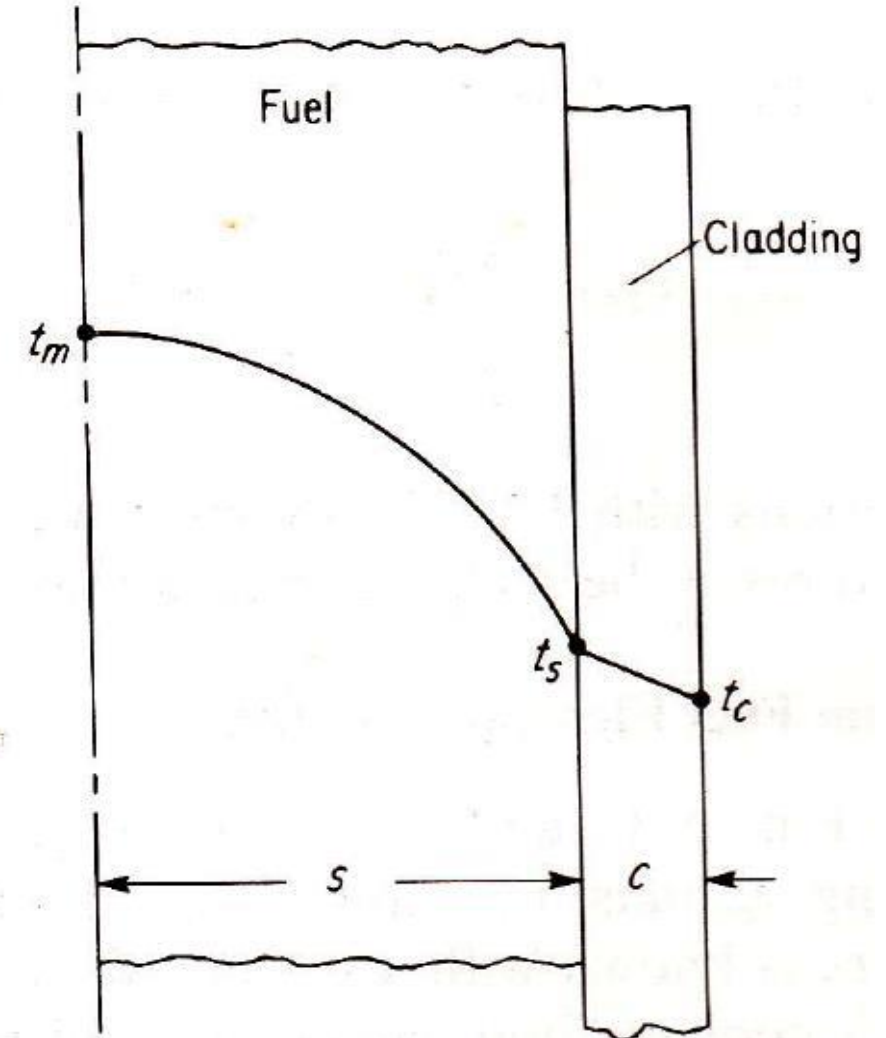
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## The Effect of Cladding

- System containing a **plate-type fuel** element of half thickness  $s$  with **cladding** of thickness,  $c$  and thermal conductivity  $k_c$ ,
- The presence of the **cladding**, an extra, though usually **small**, **resistance** to heat conduction **results** in the **decrease** of  $q_s$  for the same overall **temperature drop**
- $T_m - T_c$  were the same as  $T_m - T_s$  in the previous case





## The Effect of Cladding

- In the **steady-state** since no **heat generated** in the **cladding** material, The amount of heat **leaving surface  $s$**  same as that leaving surface  **$c$** .
- For a constant  **$k_c$**  ,  **$dT / dx$**  through the cladding is constant.
- Also neglecting resistance to heat flow at the fuel-cladding interface, we can write

$$q_s = q''' A_s = 2k_f A \frac{T_m - T_s}{s}$$

$$q_s = -Ak_c \left. \frac{dT}{dx} \right]_{clad} = Ak_c \frac{T_s - T_c}{c} \quad (14)$$



## The Effect of Cladding

$$q_s = q''' As = 2k_f A \frac{T_m - T_s}{s}$$

Solving for the temperature differences,

$$T_m - T_s = \frac{q_s s}{2k_f A} = \frac{q'''}{2k_f} s^2 \quad (15)$$

$$T_s - T_c = \frac{q_s c}{k_c A} s = \frac{q'''}{k_c} s c \quad (16)$$

Adding,

$$T_m - T_c = \frac{q_s}{A} \left( \frac{s}{2k_f} + \frac{c}{k_c} \right) = \frac{q''' s^2}{2k_f} + \frac{q''' s c}{k_c} \quad (17)$$



## The Effect of Cladding

Rearranging gives  $q_s$  in terms of the temperature

$$q_s = \frac{A(T_m - T_s)}{\left(\frac{s}{2k_f} + \frac{c}{k_c}\right)} = \frac{(T_m - T_c)}{\left(\frac{s}{2Ak_f} + \frac{c}{Ak_c}\right)} \quad (18)$$

This formula gives the rate at which heat flows through one side of the fuel plate in terms of the difference in temperature between the center and surface of the plate.

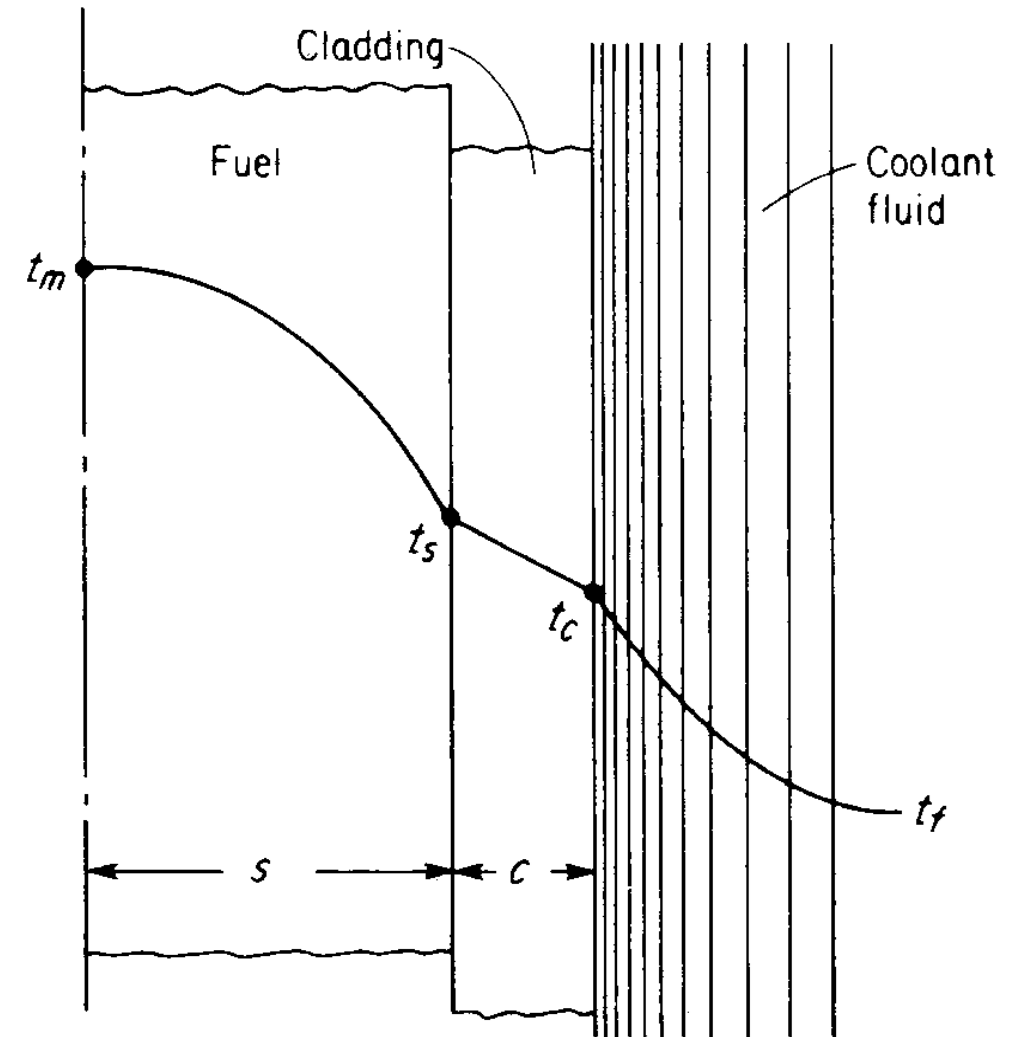
$$R = \frac{s}{2Ak_f} + \frac{c}{Ak_c} \quad (19)$$

$R$  is the total thermal resistance of the fuel and cladding.



## The Heat Transfer from Fuel Element to Coolant

- A **clad fuel plate** with **coolant fluid** passing parallel to it.
- At **steady state**, with **no heat produced in cladding or coolant**





## The Heat Transfer from Fuel Element to Coolant

If  $q_s$ , is the heat leaving the fuel element

$$q_s = q'''As = 2k_fA \frac{T_m - T_s}{s} = Ak_c \frac{T_s - T_c}{c} = hA(T_c - T_f) \quad (20)$$

Solving for the temperature differences,

$$T_m - T_s = \frac{q_s s}{2k_f A} = \frac{q'''}{2k_f} s^2$$

$$T_s - T_c = \frac{q_s c}{k_c A} s = \frac{q'''}{k_c} s c$$

$$T_c - T_f = \frac{q_s}{hA} = \frac{q''' s}{h} \quad (21)$$



## The Heat Transfer from Fuel Element to Coolant

Adding,

$$T_m - T_f = \frac{q_s}{A} \left( \frac{s}{2k_f} + \frac{c}{k_c} + \frac{1}{h} \right) = \frac{q''' s^2}{2k_f} + q''' s \left( \frac{c}{k_c} + \frac{1}{h} \right) \quad (22)$$

Which may be written as

$$q_s = \frac{(T_m - T_f)}{\left( \frac{s}{2k_f A} + \frac{c}{k_c A} + \frac{1}{hA} \right)} \quad (23)$$

$$q''' = \frac{(T_m - T_f)}{\left( \frac{s^2}{2k_f} + \frac{cs}{k_c} + \frac{s}{h} \right)} \quad (24)$$





## Interdependence of Temperature, Heat Transfer and Heat (or Neutron) Flux

**Equations (24)** may be used to explain some of the **limitations** on heat **generation** in nuclear reactors.

$$q''' = \frac{(T_m - T_f)}{\left(\frac{s^2}{2k_f} + \frac{cs}{k_c} + \frac{s}{h}\right)}$$

For any constant value of  $q'''$ , if  $T_f$  is to be kept as high as possible for good **plant thermal efficiency**,

$h$  has to be increased materially (since it affects only part of the equation) to keep the maximum fuel temperature  $T_m$  or the maximum cladding temperature  $T_s$ , from becoming **excessively highly**.



## Interdependence of Temperature, Heat Transfer and Heat (or Neutron) Flux

- *On the other hand*, if for metallurgical or other **reasons** these temperatures are **limited** to certain values,  **$h$**
- The value of  **$q'''$**  may be regulated by **changing** the **neutron flux** with the help of the **control rods**.
- In essence, there is **no limit** to the quantity of generating, so long as **adequate cooling** is provided to keep the temperatures in the system from **exceeding** their safe limits.



## Exercise 1

A *plate-type* fuel element is made of **1.5** percent enriched uranium metal. The element is **4 ft** long, **3.5 in.** wide, and **0.2 in.** thick. It is clad in **Type 304l stainless steel** with **0.005 in.** thick. The effective thermal neutron flux at the point of maximum temperature (slightly above the centre of the core) is  **$30 \times 10^{13}$**  neutrons/sec cm<sup>2</sup>. For good plant efficiency, the coolant bulk temperature at that point should be no lower than **600 °F**. Find

Given:

$$G = 180 \text{ MeV}$$

$$\text{The effective fission cross section, } \sigma_f = 364 \text{ b}$$

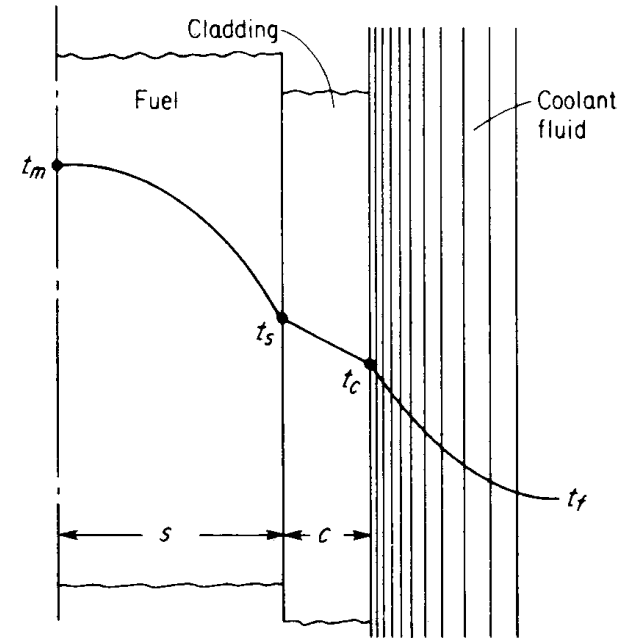
- The volumetric thermal of element at that point,  $q'''$
- The minimum value of  $h$  if the fuel temperature should not exceed 700 °F
- The maximum cladding and coolant temperatures
- The minimum value of  $h$  if the fuel temperature should not exceed 700 °F, but for neutron fluxes of  $2 \times 10^{13}$  and  $4 \times 10^{13}$



## Exercise 1 (Solution)

Solution. The physical properties of the fuel and cladding are evaluated at their respective mean temperatures. Since these are not known, the boundary temperatures are assumed as follows:

$T_m = 700^\circ\text{F}$	Given
$T_s = 660^\circ\text{F}$	Assumed
$T_c = 650^\circ\text{F}$	Assumed
$T_f = 600^\circ\text{F}$	Given



Mean fuel and cladding temperatures are **680** and **655 °F**

$\rho$  density of uranium metal at **680 °F = 18.71 gm/cm<sup>3</sup>**

$k_f$  conductivity at same temperature = **18.5 Btu/hr ft °F**

$k_c$  of 304L stainless steel at **655 °F = 11.36 Btu/hr ft °F**

From table in slide (Part 1)



## Exercise 1 (Solution)

Therefore

(a) The volumetric thermal of element at that point,  $q'''$

$$N_f = \frac{N_a \rho v(f)}{M} = \frac{(0.60225 \times 10^{24} \text{ atoms/mole})(18.71 \frac{\text{g}}{\text{cm}^3})(0.015)}{235.044 \text{ g/mole}} = 7.19 \times 10^{20} \text{ nuclei/cm}^3$$

$$q''' = GN_f \sigma_f \phi$$

$$q''' = (180 \text{ MeV})(7.19 \times 10^{20})(364 \times 10^{-24})(3 \times 10^{13}) = 1.413 \times 10^{15} \frac{\text{MeV}}{\text{sec cm}^3}$$

$$1.413 \times 10^{15} \frac{\text{MeV}}{\text{sec cm}^3} \times 1.548 \times 10^{-8} = 2.185 \times 10^7 \frac{\text{Btu}}{\text{hr} \cdot \text{ft}^3}$$



## Exercise 1 (Solution)

(b) The minimum value of  $h$  if the fuel temperature should not exceed  $700^\circ\text{F}$

$$q''' = 2.185 \times 10^7 \frac{\text{Btu}}{\text{hr} \cdot \text{ft}^3}, s = \frac{0.2}{2} \text{ in.}, c = 0.005 \text{ in.}, T_f = 600^\circ\text{F}, T_m = 700^\circ\text{F}$$

$$T_m - T_f = \frac{q''' s^2}{2k_f} + q''' s \left( \frac{c}{k_c} + \frac{1}{h} \right)$$

$$700 - 600 = \frac{\left( 2.185 \times 10^7 \frac{\text{Btu}}{\text{hr} \cdot \text{ft}^3} \right) \left( \frac{0.1}{12} \text{ ft} \right)^2}{2 \left( 18.5 \frac{\text{Btu}}{\text{hr} \cdot \text{ft} \cdot ^\circ\text{F}} \right)} + \left( 2.185 \times 10^7 \frac{\text{Btu}}{\text{hr} \cdot \text{ft}^3} \right) \left( \frac{0.1}{12} \text{ ft} \right) \left( \frac{0.005}{12} \frac{\text{ft}}{\frac{\text{Btu}}{\text{hr} \cdot \text{ft} \cdot ^\circ\text{F}}} + \frac{1}{h} \right)$$

$$700 - 600 = 41.059 + (1.823 \times 10^5) \left( \frac{0.005 \text{ ft}}{12 \left( 11.36 \frac{\text{Btu}}{\text{hr} \cdot \text{ft} \cdot ^\circ\text{F}} \right)} + \frac{1}{h} \right)$$

$$h \text{ (for } \phi = 3 \times 10^{13}) = 3488 \frac{\text{Btu}}{\text{hr} \cdot \text{ft}^2 \cdot ^\circ\text{F}}$$



## Exercise 1 (Solution)

(c) The maximum cladding and coolant temperatures

$$q''' = 2.185 \times 10^7 \frac{\text{Btu}}{\text{hr} \cdot \text{ft}^3}, s = \frac{0.2}{2} \text{ in.}, c = 0.005 \text{ in.}, T_f = 600^\circ\text{F}, T_m = 700^\circ\text{F}$$

$$700 - T_s = \frac{(2.185 \times 10^7 \frac{\text{Btu}}{\text{hr} \cdot \text{ft}^3})}{2(18.5 \frac{\text{Btu}}{\text{hr} \cdot \text{ft} \cdot ^\circ\text{F}})} (\frac{0.1}{12} \text{ ft})^2$$

$$T_s = 700 - 41.3 = 658.9^\circ\text{F}$$



## Exercise 1 (Solution)

$$t_c - t_f = \frac{q''' s}{h}$$

$$t_c - 600 = \frac{(2.185 \times 10^7 \frac{\text{Btu}}{\text{hr} \cdot \text{ft}^3})(\frac{0.1}{12} \text{ft})}{3488 \frac{\text{Btu}}{\text{hr} \cdot \text{ft}^2 \cdot ^\circ\text{F}}}$$

$$T_c = 600 + 52.3 = 652.3^\circ\text{F}$$

$$T_s = 700 - 41.3 = 658.9^\circ\text{F}$$

$$T_s = 660^\circ\text{F} \quad \text{Assumed}$$

$$T_c = 650^\circ\text{F} \quad \text{Assumed}$$

$T_c$  is the **highest coolant temperature**

At this point these calculated **boundary temperatures** are compared with the **ones assumed** at the beginning of the **solution**.

The **differences** are sufficiently **small**, and the **answers** are therefore **sufficiently accurate**.





## Exercise 1 (Solution)

(d) The minimum value of  $h$  if the fuel temperature should not exceed  $700^\circ\text{F}$ , but for neutron fluxes of  $2 \times 10^{13}$  and  $4 \times 10^{13}$

Repeat the same method used in part (b)

$$h \text{ (for } \phi = 2 \times 10^{13}) = 1783 \frac{\text{Btu}}{\text{hr} \cdot \text{ft}^2 \cdot ^\circ\text{F}}$$

$$h \text{ (for } \phi = 4 \times 10^{13}) = 6682 \frac{\text{Btu}}{\text{hr} \cdot \text{ft}^2 \cdot ^\circ\text{F}}$$

$h$  the **minimum** necessary to keep the **maximum** fuel temperature within  **$700^\circ\text{F}$** , increase rapidly with neutron flux



## Maximum Neutron Flux

This **theoretical maximum** value of **flux** is obtained by putting  **$h$  infinite** and evaluating a corresponding value of

$$T_m - T_f = \frac{q''' s^2}{2k_f} + q''' s \left( \frac{c}{k_c} + \frac{1}{h} \right)$$

$$q'''_{max} = \frac{(T_m - T_f)}{\left( \frac{s^2}{2k_f} + \frac{sc}{k_c} \right)}$$

In effect, this makes the **temperature** of the **coolant** equal to the temperature of the **cladding's outer surface,  $T_c$**



## Maximum Neutron Flux

$\phi_{max}$  can be then evaluated from  $q'''_{max}$

$$\phi_{max} = \frac{q'''_{max}}{GN_f\sigma_f}$$

To increase  $\phi$  beyond  $\phi_{max}$  for a given fuel and cladding configuration,

- It is necessary to lower  $T_f$  (with its **undesirable effect** on plant **thermodynamic efficiency**)
- To increase  $T_m$  (at the **expense** of reduced **fuel burnup** or otherwise by **fuel alloying** or using ceramic fuels), or both.



## Exercise 2

**Calculate the maximum possible neutron flux corresponding to the data from Exercise 1**

$$q''' = 2.185 \times 10^7 \frac{\text{Btu}}{\text{hr} \cdot \text{ft}^3}, \phi = 3 \times 10^{13} \text{ neutrons/sec cm}^2, s = \frac{0.2}{2} \text{ in.}, c = 0.005 \text{ in.}, T_f = 600^\circ\text{F}, T_m = 700^\circ\text{F}$$

**Given:**

$$\mathbf{G = 180 \text{ MeV}}$$

$$\mathbf{\text{The effective fission cross section, } \sigma_f = 364 \text{ b}}$$



## Exercise 2 (Solution)

**Calculate the maximum possible neutron flux corresponding to the data from Exercise 1**

$$q''' = 2.185 \times 10^7 \frac{Btu}{hr \cdot ft^3}, \phi = 3 \times 10^{13} \text{ neutrons/sec cm}^2, s = \frac{0.2}{2} \text{ in.}, c = 0.005 \text{ in.}, T_f = 600^\circ\text{F}, T_m = 700^\circ\text{F}, T_c = 652.3^\circ\text{F}$$

$$q'''_{max} = \frac{(T_m - T_{fmax})}{\left(\frac{s^2}{2k_f} + \frac{sc}{k_c}\right)}$$

$$q'''_{max} = \frac{(700 - 652.3)}{\left(\frac{(0.1)^2}{2(18.5)} + \frac{(0.1)(0.005)}{11.36}\right)} = 4.06 \times 10^7 \frac{Btu}{hr \cdot ft^3}$$

$$q'''_{max} = \frac{4.06 \times 10^7}{1.548 \times 10^{-8}} = 2.62 \times 10^{15} \frac{MeV}{sec \cdot cm^3}$$



## Exercise 2 (Solution)

$$\phi_{max} = \frac{q'''_{max}}{GN_f \sigma_f}$$

$$\phi_{max} = \frac{2.62 \times 10^{15}}{(180)(7.19 \times 10^{20})(364 \times 10^{-24})} = 5.57 \times 10^{13} \frac{\text{Neutrons}}{\text{sec} \cdot \text{cm}^2}$$



# Thank You

Stay safe!